



Technical Program Review

March 15-16, 2016

LLNL-PRES-684817

ICSBEP Accomplishments

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This work performed under the auspices of the U.S. Department of Energy by Lawrence Livermore National Laboratory under Contract DE-AC52-07NA27344

Outline

- FY-2015 accomplishments
- FY-2016 accomplishments
- FY-2016 work in progress
- NEA liaison activities
- Upcoming meetings
- Issues

Nuclear Science
September 2015

International Handbook of Evaluated Criticality Safety Benchmark Experiments



FY2015 Accomplishments – NCSP

NEA/NSC/DOC(95)03/IV
Volume IV
LEU-COMP-THERM-096

NEW

**PARTIALLY-REFLECTED WATER-MODERATED SQUARE-PITCHED
U(6.90)O₂ FUEL ROD LATTICES WITH 0.67 FUEL TO WATER VOLUME
RATIO (0.800 CM PITCH)**

Evaluator

Gary A. Harms
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Internal Reviewer
John A. Miller

Independent Reviewer

Nicolas Leclaire
Institut de Radioprotection et de Sûreté Nucléaire (IRSN)



NEA/NSC/DOC(95)03/VIII
Volume VIII

ALARM-TRAN-AIR-SHIELD-001

NEW

**NEUTRON ACTIVATION FOIL AND THERMOLUMINESCENT DOSIMETER
RESPONSES TO A BARE PULSE OF THE CEA VALDUC SILENE
CRITICAL ASSEMBLY**

Evaluators

Thomas M. Miller
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Institut de Radioprotection et de Sûreté Nucléaire



FY2015 Accomplishments – TRG

NEA/NSC/DOC(95)03/III
Volume III

IEU-SOL-THERM-005

NEW

UNREFLECTED CRITICAL DIMENSION OF AQUEOUS SOLUTION OF U(37 %)O₂F₂ IN SPHERICAL GEOMETRY

Evaluators

Tanja Kaiba
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Luka Snoj
Igor Lengar

Jožef Stefan Institute

Internal Reviewer
Igor Lengar

Independent Reviewer
Margaret A. Marshall
Idaho National Laboratory

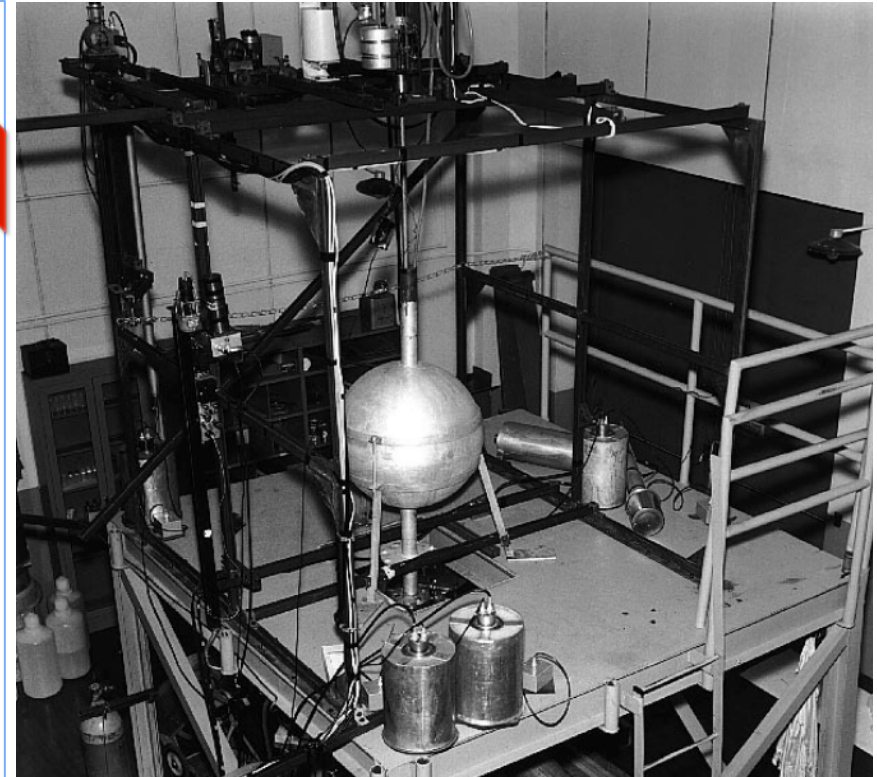


Table 3.5. Benchmark k_{eff} Values and Uncertainties (1σ).

| Model | Benchmark model k_{eff} | Uncertainty |
|------------|---------------------------|-------------|
| Simplified | 1.0041 | 0.0065 |

Table 4.1. Sample Calculation Results (k_{eff}) for the Benchmark Models.

| | k_{eff} | $\pm \sigma_{MCNP}$ | Calculation Bias (Δk_{eff}) | Relative deviation (C-E)/E ^(a) |
|-------------------------|-----------|---------------------|---------------------------------------|---|
| MCNP 6.1.0 | 0.99940 | ± 0.00006 | -0.00470 | -0.5 % |
| COG 11.1 ^(b) | 0.99943 | ± 0.00013 | -0.00467 | -0.5 % |
| KENO V.a ^(c) | 0.99934 | ± 0.00008 | -0.00476 | -0.5 % |

FY2015 Accomplishments – TRG

NEA/NSC/DOC(95)03/II
Volume II
HEU-MET-FAST-074

NEW

ORALLOY (93.2 ²³⁵U) BARE METAL ANNULI AND DISKS

Evaluator

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Internal Reviewer
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Independent Reviewers

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John T. Mihalcz
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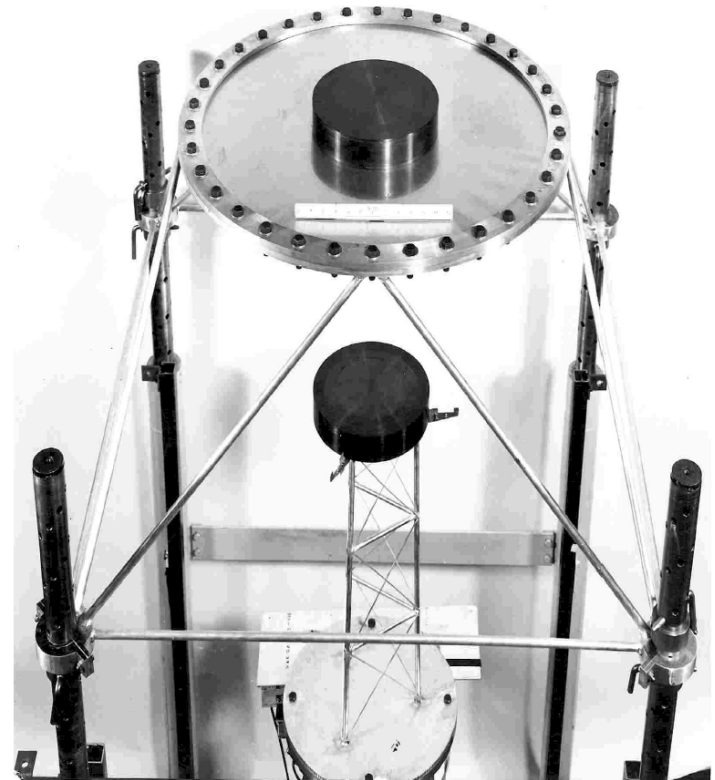


Table 4.1. Detailed Benchmark and Calculated Eigenvalues.

| Case | Calculated | | | Benchmark Experiment | | | $\frac{C - E}{E} \%$ |
|------|------------|-------|----------|----------------------|-------|----------|----------------------|
| | k_{eff} | \pm | σ | k_{eff} | \pm | σ | |
| 1 | 0.99640 | \pm | 0.00002 | 0.9988 | \pm | 0.0005 | -0.24 \pm 0.05 |
| 2 | 0.99629 | \pm | 0.00002 | 0.9979 | \pm | 0.0005 | -0.16 \pm 0.05 |
| 3 | 0.99471 | \pm | 0.00002 | 0.9970 | \pm | 0.0005 | -0.23 \pm 0.05 |
| 4 | 0.99593 | \pm | 0.00002 | 0.9975 | \pm | 0.0005 | -0.16 \pm 0.05 |

Results were calculated using MCNP6.1 with the ENDF/B-VII.1

FY2015 Accomplishments – TRG

NEA/NSC/DOC(95)03/II
Volume II
HEU-MET-FAST-077

NEW

EXPERIMENTS WITH HEU (93.14 wt.%) METAL ANNULI WITH INTERNAL GRAPHITE CYLINDER

Evaluator

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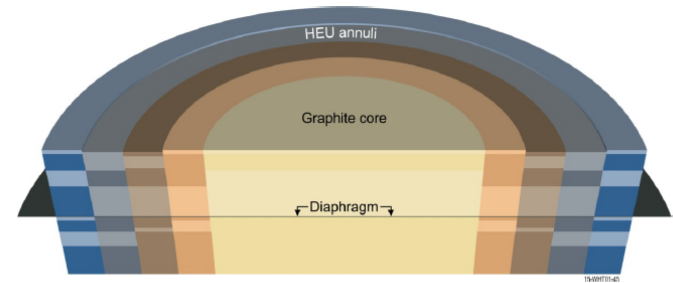


Figure 2. 3-D Figure for Core Configuration of Case 2.

Table 47. Comparison of Detailed Benchmark Model Experimental and Calculated Eigenvalues
(Case 1).

| Analysis Code | Neutron Cross Section Library | Calculated | | | Benchmark Experiment | | | $\frac{C-E}{E} \%$ |
|---------------|-------------------------------|------------|-------|----------|----------------------|-------|----------|--------------------|
| | | k_{eff} | \pm | σ | k_{eff} | \pm | σ | |
| MCNP6-1.0 | ENDF/B-VII.1 | 0.99723 | \pm | 0.00004 | 0.9981 | \pm | 0.0006 | -0.08 \pm 0.06 |
| | ENDF/B-VII.0 | 0.99735 | \pm | 0.00004 | | | | -0.07 \pm 0.06 |
| | JEFF-3.1 ^(a) | 0.99408 | \pm | 0.00004 | | | | -0.40 \pm 0.06 |
| | JENDL-3.3 ^(a) | 1.00077 | \pm | 0.00004 | | | | 0.27 \pm 0.06 |

(a) Results provided by John D. Bess from Idaho National Laboratory.

Table 48. Comparison of Detailed Benchmark Model Experimental and Calculated Eigenvalues
(Case 2).

| Analysis Code | Neutron Cross Section Library | Calculated | | | Benchmark Experiment | | | $\frac{C-E}{E} \%$ |
|---------------|-------------------------------|------------|-------|----------|----------------------|-------|----------|--------------------|
| | | k_{eff} | \pm | σ | k_{eff} | \pm | σ | |
| MCNP6-1.0 | ENDF/B-VII.1 | 0.99628 | \pm | 0.00004 | 0.9971 | \pm | 0.0006 | -0.08 \pm 0.06 |
| | ENDF/B-VII.0 | 0.99640 | \pm | 0.00004 | | | | -0.07 \pm 0.06 |
| | JEFF-3.1 ^(a) | 0.99312 | \pm | 0.00004 | | | | -0.40 \pm 0.06 |
| | JENDL-3.3 ^(a) | 1.00005 | \pm | 0.00004 | | | | 0.29 \pm 0.06 |

(a) Results provided by John D. Bess from Idaho National Laboratory.

Results were calculated using MCNP6-1.0

FY2015 Accomplishments – TRG

NEW

NEA/NSC/DOC(95)03/IV
Volume IV
LEU-COMP-THERM-067

**CRITICAL LOADING CONFIGURATIONS OF THE IPEN/MB-01
REACTOR COMPOSED OF FUEL AND MOLYBDENUM RODS**

Evaluators




Adimir dos Santos, Graciete Simões de Andrade e Silva,
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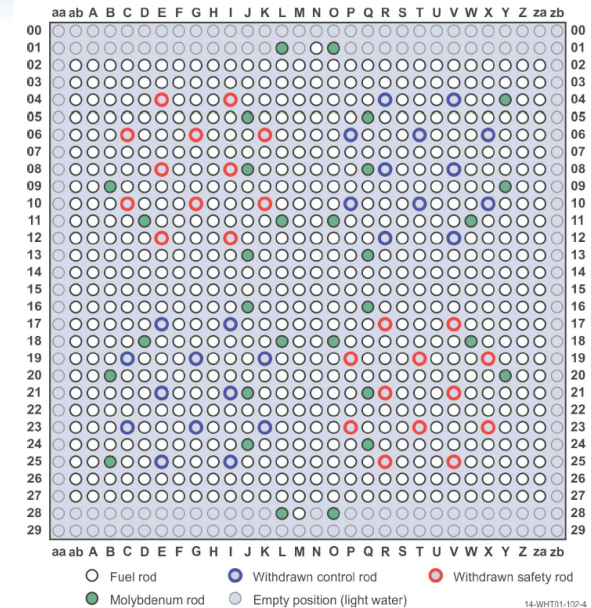


Figure 13. Experimental Core Configuration for Case 1.

Table 24. Final Values of k_{eff} and the Respective Uncertainties.

| Case | N° Molybdenum Rods | k_{eff} | Temperature |
|------|--------------------|---------------------|-------------|
| 1 | 30 | 1.0005 ± 0.0005 | 20.00 °C |
| 2 | 28 | 1.0004 ± 0.0005 | 20.00 °C |
| 3 | 24 | 1.0004 ± 0.0005 | 20.00 °C |
| 4 | 20 | 1.0005 ± 0.0005 | 20.00 °C |

Table 25. Sample Calculation Results .

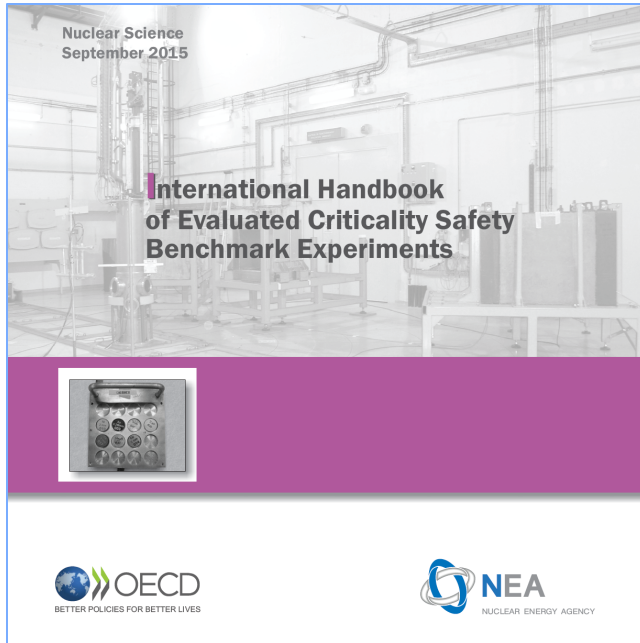
| Code (Cross Section Set) → Case Number ↓ | MCNP5 (Continuous Energy ENDF/B-VII.0) | Benchmark Value $k_{eff} \pm \sigma$ | (C-E)/E % |
|---|--|---|-------------------|
| 1 | 1.00086 ± 0.00004 | 1.0005 ± 0.0005 | 0.036 ± 0.048 |
| 2 | 1.00068 ± 0.00004 | 1.0004 ± 0.0005 | 0.028 ± 0.048 |
| 3 | 1.00082 ± 0.00004 | 1.0004 ± 0.0005 | 0.042 ± 0.048 |
| 4 | 1.00094 ± 0.00004 | 1.0005 ± 0.0005 | 0.044 ± 0.048 |

FY2015 Accomplishments – Revisions & IRPhEP

| SESSION 2: | | DISCUSSION OF MINOR REVISIONS AND APPROVED IRPhEP EVALUATIONS |
|---|--|---|
| HEU-SOL-THERM-020 | Unreflected Cylinders of Uranyl-Fluoride Solutions in Heavy Water (Revision to Level Indicator Pipe Diameter) | John Bess |
| LEU-COMP-THERM-039 | Incomplete Arrays of Water-Reflected 4.738-wt.%-Enriched Uranium Dioxide Fuel-Rod Arrays (Revision to Figure and APOLLO-MORET Calculations) | Nicolas LeClaire |
| HEU-MET-FAST-099 (ORCEF-SPACE-EXP-001) | Fast Neutron Spectrum Potassium Worth for Space Power Reactor Design Validation | John Bess |
| SESSION 3: | | DISCUSSION OF EVALUATIONS THAT HAVE BEEN REVISED TO INCLUDE ADDITIONAL DATA |
| HEU-COMP-FAST-004 (SCCA-SPACE-EXP-003) | Critical Configuration for Beryllium-Reflected Assemblies of U(93.15)O ₂ Fuel Rods (1.506-cm Pitch and 7-Tube Clusters) (Revision to Include Two New Critical Configurations) | Margaret Marshall |
| SESSION 10: | | DISCUSSION OF MINOR REVISIONS AND APPROVED IRPhEP EVALUATIONS (Continued) |
| IEU-MET-FAST-020 | The FR0 Series 1: Copper-Reflected "Cylindrical" Uranium (20 % ²³⁵ U) Metal (Revision to Include Released Data) | Dennis Mennerdahl |
| IEU-MET-FAST-022 | The FR0 Experiments with Diluted 20%-Enriched "Cylindrical" Uranium Metal Reflected by Copper (Revision to Include Released Data) | Dennis Mennerdahl |
| HEU-MET-FAST-083 | Complex Geometry Bare Orallo (93.15 ²³⁵ U) Metal Annuli Experiments | Quinton Beaulieu |

Deferred to 2017

FY2016 Accomplishments

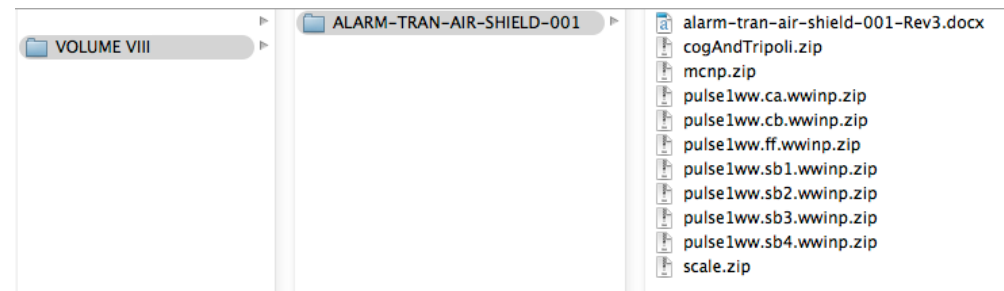


- OECD NEA provided 100 DVDs to LLNL for NCSP distribution
- 63 DVDs received and mailed out to registered NCSP users in January
- DVD size limitations are becoming a problem. 2016 solution = 2 DVDs
- DVD2 contains Volumes VIII and IX

IER161



IER126



FY2016 Work in Progress



The 2016 ICSBEP meeting will be convened jointly with IRPhEP the week of **April 18-22**, 2016, at OECD Nuclear Energy Agency (NEA) Headquarters in their new location at 46 Quai Alphonse Le Gallo, Boulogne-Billancourt, Paris. NCSP evaluations in preparation include:

- IER126, Silene CAAS Benchmark with Pb and CH₂/Cd Reflectors (T. Miller)
- IER161, BeRP/W (J. Hutchinson)
- IER285, 7uPCX with Ti Rods (G. Harms)
- Jezebel, Rev. 4 (J. Favorite)

The NCSP Manager has extended an invitation to NEA to host the ICSBEP/IRPhEP meeting in Washington, DC, the week of **October 31 - November 4**, 2017, which is the week before the ANS Winter Meeting.

Issues – Godiva IV

- No schedule for completion
- Need for revision identified in HEU-MET-FAST-086 in the disclaimer
- Additional data from DAF is available but not included in the evaluation
- Revision support IER-147, 148, 175, 268, etc.

NEA/NSC/DOC/(95)03/II
Volume II

HEU-MET-FAST-086

GODIVA-IV DELAYED-CRITICAL EXPERIMENTS AND DESCRIPTION OF AN ASSOCIATED PROMPT-BURST EXPERIMENT

DISCLAIMER:

When the Godiva IV critical assembly was refueled in 2012, additional details relevant to this benchmark became apparent. There are two revisions necessary for this evaluation to be correct:

First, the original drawing of the spindle was obsolete, which had a smaller glory hole than what is currently used in the assembly. The effect of the larger glory hole on system reactivity is negative due to the increased neutron leakage.

Second, there is a shim plate located under the safety block that alters the assumed height (and thus density) of the safety block. The presence of the shim has an important impact on the system. The density in the high importance region is currently underestimated and any neutron reflection from the shim piece is not accounted for. The corresponding correction of adding the shim and increasing the density of the safety block would have a positive effect on system reactivity.

The current evaluation is still available for use until the revised evaluation becomes available.

Additional information is provided below regarding the necessary revisions:

Spindle

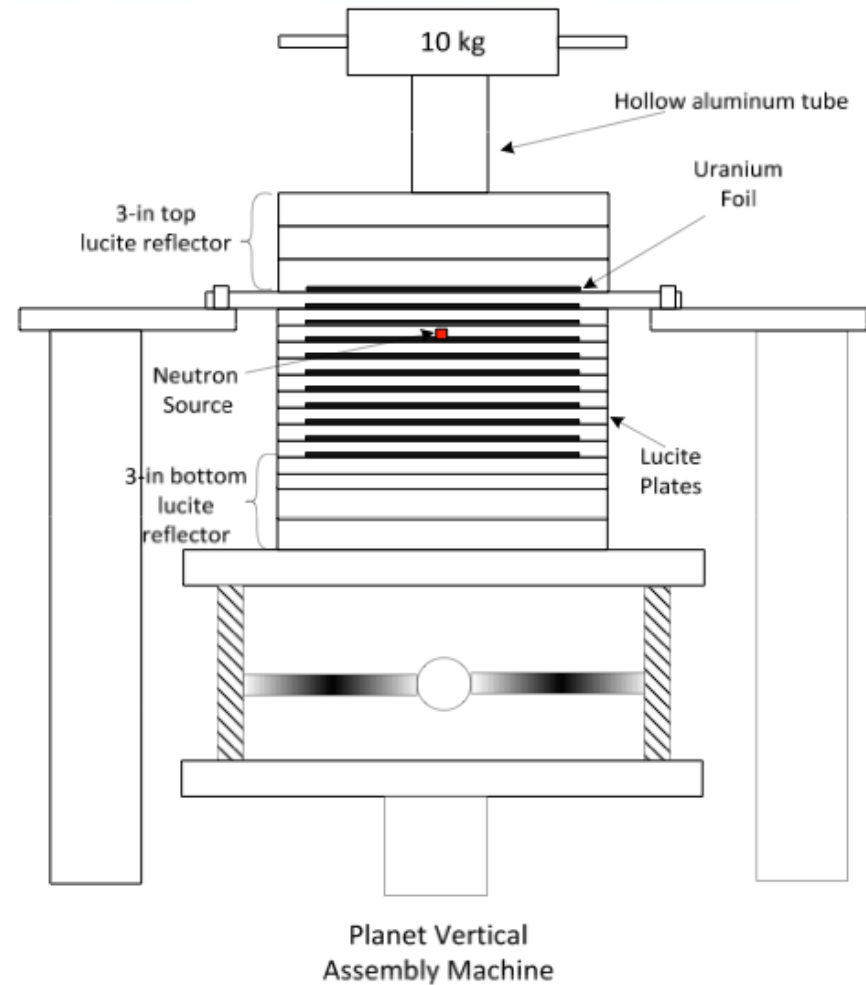
Drawing 128Y1719431, "Godiva Core Top Subassy Spindle, Mod 11B," shows a glory hole with a $\frac{1}{4}$ " (0.635 cm) inner diameter. Drawing 128Y1719515, "Godiva Core Subassy Spindle, Mod 0," is consistent with the current core configuration of Godiva IV. It shows a glory hole with an inner diameter of $\frac{9}{16}$ " (0.563", 1.42875 cm) instead. This diameter accommodates the current source holder and sample holder used for experiments on Godiva. Figure 1 shows a photograph of the current spindle with one inner fuel piece attached to it. Figures 2 and 3 show the pictorial and elevation views, respectively, of the two spindle designs.



Figure 1: Photo of current Godiva IV spindle, ca. 2005.

Issues – Class Foils in Lucite

- No schedule for completion
- Lucite plates documented in LA-UR-13-20325
- HEU-MET-THERM-032 is a very similar experiment with polyethylene instead of lucite
- HEU-MET-THERM-032 was completed by Rene Sanchez and 4 collaborators from the Jozef Stefan Institute
- Volunteers are available to assist in completion




Questions?

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HOME

ICSBEP Handbook

- History
- [Peruse Handbook on-line](#)
- Download or burn DVD images
- CD2015 VOL 1
- CD2015 VOL 2
- Request a DVD by mail
- Password request

ICSBEP Database (DICE)

- DICE User's manual
- DICE software
- Dice Data Part 1
- Dice Data Part 2

2012 New Evaluations
2013 New Evaluations
2014 New Evaluations
2015 New Evaluations
2016 New Evaluations

Integral Experiments Request (IER)

DOE Nuclear Criticality Safety Program

International Reactor Physics Evaluation Project


Evaluators

- ICSBEP Document Content and Format Guide
- ICSBEP Guide to the expression of Uncertainties

Partners

- OECD
- NEA
- CEA-Valduc
- IRSN
- VNIITF
- WPNCs


Information Preservation & Dissemination (IP&D)
A Brief History of the CSBEP, ICSBEP and NCSP



Mr. Dae Chung
Principal Deputy Assistant Secretary
United States Department of Energy

In 1992, the Criticality Safety Benchmark Evaluation Project (CSBEP) was founded under the auspices of US DOE Office of Defense Programs by Mr. Dae Chung with criticality safety experts participating from across the US DOE Complex:

- Argonne National Laboratory
- Hanford
- Lawrence Livermore National Laboratory
- Oak Ridge National Laboratory
- Sandia National Laboratories
- Savannah River National Laboratory
- Bettis Atomic Power Laboratory
- Idaho National Laboratory
- Los Alamos National Laboratory
- Pacific Northwest National Laboratory
- Rocky Flats Plant
- Y-12 Plant



Dr. Jerry McKamy
Manager
Nuclear Criticality Safety Program
Director
Facilities Operations Division
United States Department of Energy

In 1994, the CSBEP welcomed its first international participants from France, Hungary, Japan, the Russian Federation, and the United Kingdom.

In 1995, to further enhance international participation, the DOE allowed the CSBEP to become an official activity of the Organization for Economic Cooperation and Development (OECD), Nuclear Energy Agency (NEA), Working Party on Nuclear Criticality Safety (WPNCs), and the name was changed to the International Criticality Safety Evaluation Project (ICSBEP).

In 1997, the Nuclear Criticality Safety Program (NCSP) was formally established by DOE under the auspices of the Office of Defense Programs.

Today, the ICSBEP remains an important element of the US DOE NCSP as described by Dr. Jerry McKamy in the NCSP *Mission and Vision*. Current NCSP activities including ICSBEP participation are described in the *Five-Year Execution Plan*.

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The [Lawrence Livermore National Laboratory](#) maintains this website.

Updated: Friday, March 4, 2016